

NEK



TECHNICAL SPECIFICATION

Handbook for Crack flaw evaluation on Core Barrel Welds

Safety Related

Specification Number

SP-ES1504

Rev. 0

KRŠKO NUCLEAR POWER PLANT

June, 2026

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1. BACKGROUND

- 1.1 NEK is a 2-loop PWR power plant of the Westinghouse design with an original thermal power of 1882 MWt, uprated in year 2000 to 1994 MWt. It has been in commercial operation since 1983 and currently is running fifth inspection interval based on ASME Section XI, 2017 Edition. Fifth Inspection Interval started in July 2022 and will end in July 2032. Since the beginning of 2024, the nuclear power plant has been operating under its first extended operating life (for an additional 20 years).
- 1.2 With this technical specification (TS), NEK intends to obtain a procedure with acceptance criteria for core barrel welds potential MRP-227 relevant inspection findings, which will enable NEK to prepare a justification for continuous operation.
- 1.3 The general standards for determining relevant conditions in reactor internals inspections are provided in MRP-228:
 - Welds – Cracks, or indications that exhibit characteristics of cracking, are considered relevant.
 - Components – Cracking or other significant degradation that could impair the ability of the component to perform its design function is considered relevant.

Once a relevant condition has been identified, evaluation is required to assess the ability of the degraded component to continue to perform the design function without interfering with the function of the system.

- 1.4 The Westinghouse-designed core barrel (CB) is large, thick-walled cylinder, which support the core and other reactor internals components. A CB consists of austenitic stainless-steel plates rolled and welded into three cylindrical shell sections: the upper, middle, and lower barrel cylinders. The shell sections are welded together with girth welds to form the complete barrel. In both designs, a flange is welded to the upper end of the barrel, while either another flange or a thick plate or casting is welded to the bottom end. The barrel provides a passageway for the reactor coolant flow, directing flow into the core and directing the core outlet flow to the outlet nozzles.
- 1.5 The early operating experiences (OEs) related to pressurized water reactor (PWR) CB degradation are provided in Appendix A of MRP-227. Two Combustion Engineering (CE) plants (St. Lucie Unit 1 and Millstone Unit 2) experienced a substantial amount of cracking in the 1980s. In both cases, additional flaw growth was mitigated by drilling out the ends of the cracks. Both plants have operated safely and reliably in this condition for decades since the corrective actions were implemented.
- 1.6 During the Spring 2018 MRP-227-A inspection of the core support barrel (CSB) welds at St. Lucie Unit 1, a CE designed PWR, visual indications of cracking were observed [6], [7]. These indications were located adjacent to the middle girth weld (MGW) and middle axial weld (MAW) of the CSB, in and near the high neutron fluence core beltline region. A supplemental volumetric inspection of the area confirmed that the indications were flaws with some depth and that none of the flaws had propagated through-wall. The indications were dispositioned by an engineering evaluation, and St. Lucie Unit 1 returned to power for another cycle of operation.

A reinspection of the CSB welds was performed during the Fall 2019 refueling outage. The evaluation of the reinspection results concluded that the reported flaws are acceptable for the remainder of the 10-year ISI interval. The MAW and MGW were re-inspected in 2025 which reported additional MAW indications, growth of previously inspected MAW flaws, and additional MGW indications. The flaws were evaluated to be acceptable for one cycle of operation. An apparent cause of flaws in the CSB has not yet been confirmed.

- 1.7 During the Fall 2022 10-year ISI inspection of the CB welds at H.B. Robinson Unit 2, a Westinghouse-designed PWR, visual inspections and extent of condition ultrasonic inspections (consistent with MRP-227 [2], [19]) of the CB found a total of five circumferentially oriented cracks in the heat affected zone (HAZ) of the UGW [24]. All five flaws were observed on the inner diameter (ID) of the UGW. The visual examinations were confirmed by ultrasonic inspections, which were also used to determine the length and depth of the indications. Two of the flaws were 90% or greater through wall (TW). All but one of these indications were determined to be acceptable by engineering evaluation for one cycle of continued operation. For the flaw that could not be accepted by evaluation, the utility owner installed two crack tip arrest holes to mitigate any cracking during continued operation.

In the Fall 2024 outage, the CB UGW was reexamined, with no new indications identified. The 4 unmitigated indications were re-measured and did not show measurable growth with EVT-1 or UT. Crack arrest holes were installed on the right side of Flaw 4 and on Flaw 3, and samples were captured for metallurgical analysis. Plugs were installed in the new crack arrest holes to prevent bypass flow. After the 2024 outage efforts, 3 of 5 CB UGW flaws discovered in 2022 were considered to be mitigated. All crack arrest holes and plugs were justified through a second period of operation (i.e. 80-year plant life).

- 1.8 During the Spring 2025 outage inspection of the CB welds at Salem Unit 1, a Westinghouse designed PWR, visual inspections and extent of condition ultrasonic inspections of the CB found six circumferentially oriented indications and four off-axis indications near the UGW. The indications were noted as part of extent of condition inspections in response to discovery of loose parts (later identified as being bolts and bolt parts from the replacement thermal shield flexures) on the lower core plate. Subsequent ultrasonic (UT) examinations were performed and confirmed that the indications were flaws with some depth and that none of the flaws had propagated through-wall. All but two of these indications were determined to be acceptable by engineering evaluation for one cycle of continued operation. Those two flaws were mitigated during the same outage by excavation. Two of the indications were detected by EVT-1 but not by UT and two were detected only by UT.
- 1.9 During the Fall 2025 inspection at Diablo Canyon Unit 2, a lower girth weld (LGW) indication was observed from an EVT-1 inspection. The length of this indication did not require inspection of expansion components and a UT inspection was not performed. Continued operation was justified based on flaw tolerance evaluations developed in preparation for the examination.
- 1.10 NEK performed MRP-227 inspections during outages 2021 and 2022 to fulfill commitments from AMP which is developed in accordance with NUREG-1801, Rev.2; GALL; XI.M16A PWR VESSEL INTERNALS and outlined in program TD-20; Program nadzora notranjih delov reaktorske posode. This included EVT-1 inspections of following CB welds (Figure 1):

- W3; Core Barrel Assembly; Upper Flange Weld (UFW) (EVT-1)
- W4; Core Barrel Assembly; Lower Girth Weld (LGW) (EVT-1)

- 1.11 For the inspection in 2021, a "Summary of analytical input data for justifications of one cycle of the lower internal parts of the Krško MRP-227 reactor vessel" was developed for UFW and LGW based on the methodology described in MRP-376, which, however, is outdated according to PWROG-19044, Rev.1 and should be used for informational purposes only.

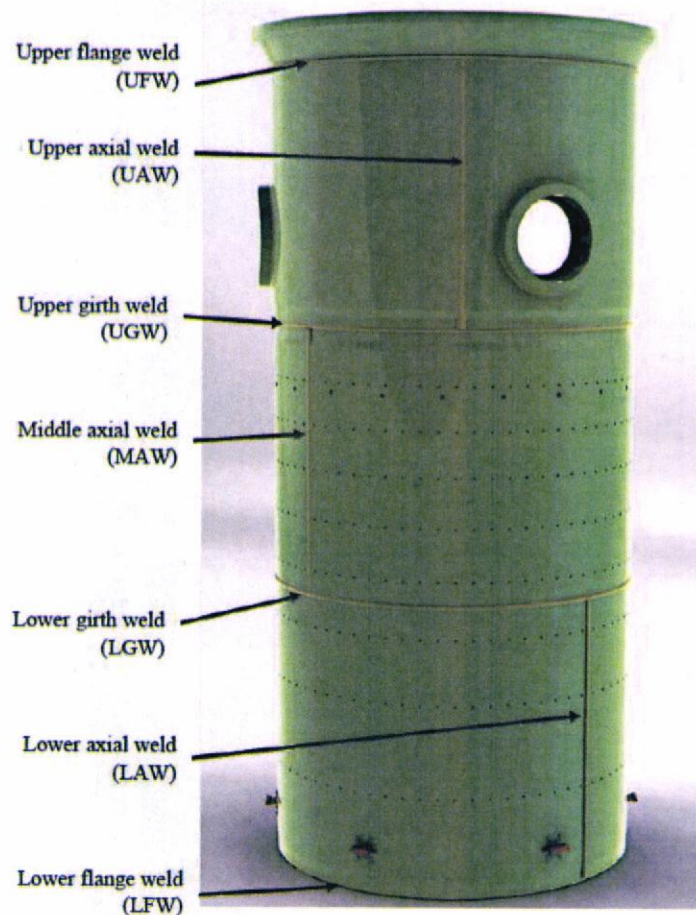


Figure 1: General Schematic of a Westinghouse Core Barrel

- 1.12 During the April 2027 outage, NEK planned to inspect the UGW (W3a) to meet the requirements of MRP 2024-004. For a planned inspection in accordance with MRP-227, Rev. 2-A, it is necessary to develop the handbook with the NEK specific acceptance criteria for potential Core Barrel inspection relevant findings (indications, cracks).

Development of Plant Specific Acceptance Criteria is important due to following (Ref. 6.14):

- Acceptance criteria developed on a plant basis will typically be less limiting than generic acceptance criteria.
- Preparing acceptance criteria prior to the inspection prevents a potentially long delay required to develop the acceptance criteria.

- US NRC approved flaw acceptance criteria methodology provided in WCAP-17096-NP-A.
- There is currently no approved generic acceptance criteria developed that complies with current WCAP-17096-NP-A guidance.

2. SCOPE OF WORK

- 2.1 The scope of work is to develop the NEK specific acceptance criteria which will enable NEK to prepare a justification for continuous operation (JCO) with no additional delay in outage timeframe. The process to develop the flaw acceptance criteria (see WCAP-17096-NP) is to first determine the flaw length that cannot be exceeded based on faulted, normal/upset, and flow induced vibration stresses. This allowable flaw length is then adjusted (shortened) by the amount of calculated growth projected between the end of extended life (end of 2043) and the inspection date April 2027.

The change in resistance to fracture of the RV core barrel welds must be correlated to the accumulated fluence at each weld location. Welds that are subject to low fluence are considered to have a high degree of resistance to fracture. Correspondingly, those welds subject to high fluence have lower resistance to fracture:

- (1) Upper Girth Weld (UGW)
- (2) Upper Flange Weld (UFW)
- (3) Lower Girth Weld (LGW)
- (4) Upper Axial Weld (UAW)
- (5) Lower Axial Weld (LAW)
- (6) Lower Flange Weld (LFW)

The tables and charts provided therein shall allow the evaluation of any relevant indication discovered in the prescribed regions with simplified methods (without complex fracture mechanics calculations). All necessary user-friendly procedures for preparing a JCO must be part of the handbook.

- 2.2 A Chapter in the handbook on the application of the handbook with an additional power point presentation.
- 2.3 The technical report shall be the exclusive property of NEK, which shall retain full ownership rights. NEK may use, reproduce, and distribute the report without any limitations or restrictions.

3. SAFETY CLASSIFICATION OF WORK

- 3.1 Service within the scope of this specification is defined as Safety Related »SR«, due to the safety classification of the equipment identified in 6.0 of this specification.

4. INPUT DATA

4.1 Methodology Inputs

EPRI Document MRP-227, Rev. 2-A provides a detailed description of industry practices for PWR internals inspections to help ensure the materials maintain safe functionality over the extended life of the plant.

WCAP-17096-NP is a Westinghouse report that identifies analytical methodologies and data requirements for materials reliability program (MRP) reactor internals I&E as well as discussion of evaluation procedures. This document is used as input for background information in the W3 and W4 evaluations.

PWROG-19044-NP is a Westinghouse report that is designed specifically to address the CB in Westinghouse- and CE-designed PWRs. The primary purpose of this report is to serve as a reference for program engineers and managers in planning inspection campaigns of the CB and in responding to inspection indications observed during examinations of the CB.

4.2 Krško NPP Data Input

Chapter 5 of the Krško USAR provides details about design basis loading, which indicates leak before break is applicable, and therefore all large primary loop ruptures have been eliminated. In addition, EFPY at the end of Cycle 34 was 37,28 EFPY and a conservative estimate for EOC 35 (outage April 1, 2027) is 38,70 EFPY. Conservative estimation of EFPY for end of extended plant life (until end of 2043) is 54 EFPY.

NEK documents:

1. NEK USAR, Rev. 32, "Updated Safety Analysis Report"
2. SSR-NEK-5.1, "RSG Design Transients Specification", Rev. 3, Nov. 1999
3. WENX 13-03, Rev.3, Analysis of Capsule T from the NEK Reactor Vessel Radiation Surveillance Program
4. AMP-TA-04, "RCL Piping and RCL Components Fatigue", Rev.6
5. AMP-TA-06, "Environmental Fatigue Evaluations per NUREG/CR-6260", Rev. 7
6. AMP-TA-07, Reactor Vessel Beltline Fluence Evaluation, Rev. 4
7. AMP-TA-08, "Reactor Vessel Irradiation Embrittlement", Rev. 6
8. TD-20, Program nadzora notranjih delov reaktorske posode, Rev.5
9. ED-5, Reactor Vessel Irradiation Surveillance Program, Rev. 4
10. TD-2E/5, "Inservice Inspection Program - The 5th Inspection Interval", Rev.1
11. NEK ESD-TR-20/25, "NPP Krško Fatigue Transients Evaluation - Cycle 34"
12. WCAP-18756-NP, Ex-Vessel Neutron Dosimetry Program for Krsko During Cycle 26 Through Cycle 31

4.3 Dimensional Information

The following drawings are used as input to provide information about the areas included in the W3, W3a and W4:

Krško USAR Rev. 32

Krško Drawing 6119E42, Rev. 1, Reactor Pressure Vessel with Internals

Krško Drawing 113E833, SHEET 1 of 2, Rev. 5, Core Barrel Weldment

Krško Drawing 113E983, SHEET 1 of 4, Rev. 5, Upper Core Barrel

Krško Drawing 113E983, SHEET 2 of 4, Rev. 5, Upper Core Barrel

Krško Drawing 113E870, SHEET 2 of 4, Rev. 5, Lower Core Barrel

Krško Drawing 271C095, Rev. 1, Core Barrel Flange

4.4 Materials

All base materials for the core barrel components are considered to be Type 304 Stainless Steel (Table 3-3 on page 3-28 of MRP-227). Chapter 4 of the USAR, while not specifically addressing the core barrel, lists the reactor vessel internal structural components to generally be constructed of Type 304 stainless steel. The core barrel drawings that were provided as inputs do not show material designations with the exception of drawing 271C095 which confirms that the upper core barrel flange is made of ASTM A182 Grade F304, which is a material specification for type 304 stainless steel forgings.

Therefore the guidance from MRP-227 is considered accurate for the remainder of the core barrel components.

- 4.5 Additionally, a Contractor shall define the list of all necessary additional inputs (NEK procedures, calcnotes, other documents) needed for performing the described scope of work. NEK will inform Contractor on availability of information sources and will inform Contractor which of the stated sources can be delivered to Contractor and under which terms. NEK may decide that to some of the sources a Contractor will have access only at NEK site.

- 4.6 The data from NEK plant computer shall be prepared by NEK and will be in the following format:

date/time, sensor 1, sensor 2, sensor 3

1/1/2021 12:01:00, value for sensor 1, values for sensor 2, value for sensor 3

5. APPLICABLE WORK CONTROL PROGRAM

- 5.1 A formal work control program is not applicable.

6. APPLICABLE CODES, STANDARDS, AND CRITERIA FOR THE WORK

The following codes, regulations and standards shall be used as acceptance criteria for independent evaluation process:

- 6.1 10 CFR 50, Appendix A, "General Design Criteria for Nuclear Power Plants", US NRC
- 6.2 10 CFR 50, Appendix B, "Quality assurance criteria for nuclear power plant and fuel reprocessing plants", US NRC
- 6.3 10 CFR 21, "Reporting Of Defects and Noncompliance", US NRC
- 6.4 10 CFR 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants", US NRC
- 6.5 NUREG-1801 Rev. 2, GALL Report, US NRC

- 6.6 LR-ISG-2011-05 – “Final License Renewal Interim Staff Guidance LR-ISG-2011-05 - Ongoing Review of Operating Experience”
- 6.7 NUREG-1800 Rev. 2, Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants, US NRC
- 6.8 ASME B&PV Code, Section III, Division 1, NB for Class 1 components, 1971 edition with all addenda up to and including Winter 1972
- 6.9 ASME B&PV Code, Section XI, Division 1, 2017 Edition
- 6.10 NEI 03-08 Guideline for the Management of the Material Issues, Rev. 4, October 2020
- 6.11 MRP-227, Pressurized Water Reactor Internals Inspection and Evaluation Guidelines, Revision 2-A, May 2025
- 6.12 MRP-228, Inspection Standard for Pressurized Water Reactor Internals—2023 Update, Rev. 5, December 2023
- 6.13 WCAP-17096-NP, Reactor Internals Acceptance Criteria Methodology and Data Requirements, Rev. 3
- 6.14 PWROG-19044-NP, Core Barrel Weld Degradation Response Option Playbook, Materials Committee PA-MS-1986, April 2026
- 6.15 IAEA SSG-48, “Ageing Management and Development of a Program for Long Term Operation of Nuclear Power Plants”, IAEA 2018
- 6.16 INTERNATIONAL ATOMIC ENERGY AGENCY, Aging Management for Nuclear Power Plants: International Ageing Lessons Learned (IGALL), Safety Report Series No.82, Rev. 2, Vienna, 2024

7. AFFECTED SYSTEM

- 7.1 Reactor Coolant system. It is classified as Safety Related (SR).

8. IDENTIFICATION OF AFFECTED EQUIPMENT

- 8.1 The affected component is Core Barrel that is part of Reactor Vessel Internals (ASME Class 1).

9. REVIEW AND/OR VERIFICATION OF THE WORK

- 9.1 The Contractor is required to provide preliminary reports, which will be reviewed by the NEK. The NEK will send the comments to the Contractor, who will issue a final version of the reports after the process of commenting is finished. There could be more than one iteration to finalize the report.
- 9.2 The Contractor shall provide a copy of all reference documents (pages) used in either calculations or assumptions as an appendix to complete work package. This will be needed for more expedient NEK review process.

10. FORMAT OF EVALUATION REPORT

- 10.1 Evaluation report shall be in format required per Contractor Quality Program, as well as procedures, defined requirements for required safety-related analysis and documentation of work.
- 10.2 All submitted documents shall bear at least following identification:
- Date of issue
 - Document number
 - Revision number
 - NEK's Purchase Order (Contract) Number
 - NEK's Specification Number
 - Contractor name, location and employee responsible for the preparation of the document
 - Contractor's order number

All document deliverables shall be submitted in two versions as a minimum: PRELIMINARY for NEK review, and FINAL version to be approved by NEK. All documents for review shall be delivered to NEK in soft copy (pdf files structured with bookmarks and active cross reference links) reviewed and approved by Contractor first. All final documents shall be delivered in addition to pdf files also as one hard copy and one soft (editable) copy:

- Word Processing: Word ©
- Spreadsheet: EXCEL©
- Computer-aided Drafting: AutoCAD©

Hard copies shall be good qualities full-size, reproducible, sharp, direct-contact prints of the Contractor's original drawings. They shall be provided together with cover form (DIW) in accordance with NEK procedure ADP-1.2.116.

The documents shall be formatted in files and printed as hardcopies in A4, A3, or A2 sizes only. The exceptions could be related only to the revisions or mark-ups of the existing NEK drawings that could be formatted in different (larger) formats.

Documents will be reviewed by NEK within 10 working days after receipt.

11. SCHEDULE REQUIREMENTS

- 11.1 Proposal required response date is 30. 07. 2026.
- 11.2 Overall Schedule Requirements for project deliverables (milestone dates) are following:

<i>Description</i>	<i>Milestone</i>
Preliminary Handbook Crack flaw evaluation on Core Barrel Welds	November 30, 2026
Final Handbook Crack flaw evaluation on Core Barrel Welds	December 31 , 2026

12. STATUS REPORTING REQUIREMENTS

- 12.1 There are no special status reporting requirements.

13. WORK TO BE PERFORMED OR INFORMATION TO BE PROVIDED BY NEK

- 13.1 The NEK Project Manager will provide the overall project coordination from NEK side. In the first phase, NEK will perform detailed review of bidder document "Detailed Work Plan", which is the subject of coordination before contract assignment.
- 13.2 During the evaluation phase, NEK will provide all available plant documentation and plant design reference documents which will be needed.
- 13.3 NEK will also coordinate and perform review of all generated documents (technical reports) developed as an interim or final product.
- 13.4 NEK will support contractor experts with technical specialists on area where needed to describe, support, or clarify opened technical issues.
- 13.5 NEK will prepare all required input data, for the analysis performed, which also includes measurements from plant computer which are at NEK disposal in a format described in chapter 4.3.

14. CHANGES OF WORK SCOPE

- 14.1 The Contractor shall identify any scope changes that could cause an impact the Contractor's cost or schedule of the project by the issuance of a Contractor Change of Work Scope Request. The Contractor shall not proceed with a change in the scope of the Work until written approval has been authorized by NEK. It is the Contractor's obligation to notify the NEK Responsible Commercial Person, writing of the noted scope change and it is the responsibility of the NEK Responsible Commercial Person to attempt to respond within 15 working days of the acceptability of the Contractors scope change request. Unauthorized Work will not be reimbursed by NEK.

15. DELIVERABLES TO BE PROVIDED BY THE CONTRACTOR

- 15.1 The Contractor will be required to furnish the following deliverables according to the schedule outlined below:

Deliverable Item	Number of Copies
Preliminary Handbook Crack flaw evaluation on Core Barrel Welds	1 (Adobe pdf)
Final Handbook Crack flaw evaluation on Core Barrel Welds	1 (Adobe pdf)
Presentation on the use of the Handbook, example	.pptx

15.2 Other types of documents to be provided by the Contractor:

- a. Schedule update.
- b. Copy of all used references, calculations, detailed evaluation worksheets and other documents essential for review, verification or approval of work been performed. Deliverables should be sent to the NEK Project Manager according to the schedule.

15.3 NEK will provide comments/approval on the preliminary work reports within two weeks after delivery.

15.4 Contractor shall resolve all potential open items from NEK comments and issue the Final work report within three weeks after receipt of NEK comments.

Word Processor: Microsoft Word 97, Excel 97 (For figures and tabulated data)
 Paper: A4
 Margins: Top, Bottom, Left, Right – 2.54 cm (1 inch)
 Font: Calibri 11 pt (text)
 Calibri 14 pt Bold (Chapter Title – i.e. Chapter or Chapter 2.0)
 Times New Roman 12 pt Bold (Subchapter Title – i.e. Subchapter .1, or Subchapter 5.2)

15.5 All documentation shall be written in English.

16. DOCUMENTS AND RECORDS

- 16.1 All the final and interim documents in connection with the work implementation shall be delivered to NEK in the prescribed format in time.
- 16.2 Verification and validation records shall be in accordance with applicable Contractor procedures (QA manual).

17. ORGANIZATIONAL CONTACT

17.1 The Contractor shall coordinate all technical and schedule matters with PM.

17.2 Commercial items are to be addressed to the NEK Commercial Department representative.

NEK representatives:

NEK Project Manager (PM):

Stanko Manojlović

Phone: +386 7 4802 527

E-mail: stanko.manojlovic@nek.si

The Contractor shall coordinate all technical and schedule matters with the NEK Independent Review Evaluation Responsible PM and Responsible Engineer.

17.3 NEK Responsible Analysis Engineer:

Janez Župec,

Phone: ++386 7 4802 922

E-mail: janez.zupec@nek.si

NEK Responsible ISI Engineer:
Justin Božič
Phone: ++386 7 4802 656
E-mail: justin.bozic@nek.si

- 17.4 Commercial items are to be addressed with the NEK Commercial Department representative. At NEK the following are commercial contact:

NEK Commercial Representative: TBD

18. CONTRACTOR TECHNICAL APPROACH TO WORK

- 18.1 The Contractor is fully responsible for organization and performance of work in the scope and requirements of the given specification. Plan of the Contractor technical approach and developed schedule should be agreed and submitted to the NEK before the start of the project work.

19. ACCESS TO CONTRACTOR FACILITY AND DOCUMENTS

- 19.1 For the purpose of reviewing the quality and the amount of the work being performed, a Contractor will provide access to Contractor's or to its Authorized Sub-contractor's working process to NEK. A Contractor will introduce to NEK Contractor's/Sub-contractor's personnel assigned to contracted work.

20. SUBCONTRACTED WORK

- 20.1 The Contractor could subcontract any portion of the Work. The subcontracted work could not be subcontracted without the written approval of the NEK Responsible Manager. Subcontractors will be required to be technically qualified to the satisfaction of NEK criteria.

21. SUPPORT OF LICENSING PROCESS AT SLOVENIAN NUCLEAR SAFETY ADMINISTRATION

- 21.1 The Contractor shall be responsible to provide NEK all necessary support for licensing process (meetings, presentations, additional justification) as requested by Slovenian Nuclear Safety Administration (SNSA).

22. QUALITY ASSURANCE REQUIREMENTS

- 22.1 All services shall be performed in accordance with the Supplier's Quality Assurance Program that complies with 10CFR50, App. B and has been previously evaluated and accepted by NEK. The Supplier shall implement and maintain this program while carrying out the requirements of this specification; all proposed changes to the program shall be submitted to and approved by the NEK prior to implementation.

If a controlled copy of the latest revision of the Supplier's QA Manual has not been previously sent to NEK, the Supplier shall submit a copy with the rest of the bidding documentation.

- 22.2 The Contractor has the responsibility for QA activities for all work pursuant to this Specification. All technical and quality requirements shall be met. All work performed by Subcontractor(s) shall be subject to the technical and quality requirements of this Specification as well. The Contractor shall be responsible for all subcontracted activities.
- 22.3 A record system shall be established and maintained by the Contractor to provide documentary evidence of the quality of performed activities. Records shall, as a minimum, identify the Purchaser's name, Purchaser's order number, inspector or data recorder, inspection date, type of observation, procedures used, results, acceptability, and action taken with any deficiencies noted. Records of inspection shall also include identity of drawings and procedures utilized, along with the revision level. All quality verification records, procedures, and qualifications shall be identifiable to the activity involved.

23. NEK PROPRIETARY DATA

- 23.1 NEK has a proprietary interest in all the drawings, designs, specifications, documents, information, or know-how that may be furnished pursuant contract execution. Also, NEK has a proprietary interest in any know-how, improvement, discovery, or invention which may be made, developed, or conceived in the performance of work here under or which may arise or result there from (hereinafter collectively referred to as the "Information"). All such information shall be proprietary to the NEK. The right to use of all such Information shall be transmitted to the Contractor only for its personnel use and shall be entirely restricted to the performance of the Contract and subject to the confidentiality provision.